Nuclear Engineering and Design 110 (1989) 423–424
North-Holland, Amsterdam



# AUTHOR INDEX TO VOLUME 110

Adams, S.K., see Husseiny 110 (1988) 277
Almenas, see Di Marzo 110 (1988) 107

Balourdet, M., see Steiner 110 (1988) 361
Bhanti, D.P., S.V. Malvankar, P. Kotrappa, S. Somasundaram, B. Raghunath and A.M. Curtay, Aerosol release factor for Pu as a consequence of ion exchange resin fire in the process cell of a fuel reprocessing plant 110 (1988) 117
Brandes, K., Assessment of the response of reinforced concrete structural members to aircraft crash impact loading 110 (1988) 177
Brückner-Foit, A., Th. Schmidt and J. Theodoropoulos, A comparison of the PRAISE code and the PARIS code for the evaluation of the failure probability of crack-containing components 110 (1988) 395
Bruna, R. and J.D. Riera, Towards the simultaneous generation of the three components of the seismic acceleration on rock surface 110 (1988) 153
Bubb, H., System ergonomics as an approach to improve human reliability 110 (1988) 233

Callow, R.A., see Fletcher 110 (1988) 313
Curtay, A.M., see Bhanti 110 (1988) 117

Dean, E.M., see Lehto 110 (1988) 47
Desmond, T.P., see LaFramboise 110 (1988) 131
Dhir, V.K., see Wang 110 (1988) 61
Di Marzo, M., K.K. Almenas, Y.Y. Hsu and Z. Wang, The phenomenology of a small break LOCA in a complex thermal hydraulic loop 110 (1988) 107
Dobbernack, R., see Kordina 110 (1988) 177

Fletcher, C.D. and R.A. Callow, Long-term recovery of pressurized water reactors following a large break loss-of-coolant accident 110 (1988) 313
Fryer, R.M., see Lehto 110 (1988) 47
Furuta, K., see Kataoka 110 (1988) 99

Golay, M.W., see Seong 110 (1988) 33

Hampl, N.C., see Schuëller, 110 (1988) 221
Hara, F., Seismic-load-induced human errors and countermeasures using computer graphics in plant-operator communication 110 (1988) 265
Hayashi, T., see Takizawa 110 (1988) 385
Holmes, J.W., see Husseiny 110 (1988) 277
Hsieh, C.K. and H. Shang, Solution of boundary value heat conduction problems with variable convective coefficients by a boundary condition dissection method 110 (1988) 17
Hsu, Y.U., see Di Marzo 110 (1988) 107
Husseiny, A.A., Z.A. Sabri, D. Packer, J.W. Holmes, S.K. Adams and R.J. Rodriguez, Operating procedure automation to enhance safety of nuclear power plants 110 (1988) 277
Jehlicka, P., Seismic design concept using methods of probabilistic structural mechanics 110 (1988) 247

Kataoka, K., K. Furuta, Y. Oka and S. Kondo, Neutronic and thermodynamic feasibility of $^{233}UF_6$–$^{237}NpF_6$ fueled transmutation reactor 110 (1988) 99
Kendoush, A.A., Measurements of neutron induced nucleation 110 (1988) 349
Kobayashi, T., Probability analysis of an aircraft crash to a nuclear power plant 110 (1988) 207
Kondo, S., see Kataoka 110 (1988) 99
Kordina, K., and R. Dobbernack, Preventive fire protection in nuclear power plants 110 (1988) 177
Kotrappa, P., see Bhanti 110 (1988) 117
Krutzik, N.J., Reduction of the dynamic response by aircraft crash on building structures 110 (1988) 191

LaFramboise, W.L. and T.P. Desmond, Nonlinear seismic analysis of a graphic reactor core 110 (1988) 131
Lehto, W.K., E.M. Dean and R.M. Fryer, Reactor safety implications of pump run-up tests in EBR-II 110 (1988) 47
Leung, L.K.H., see Snoek 110 (1988) 299
Liemersdorf, H. and L. Sütterlin, Concepts and design criteria for the protection of nuclear facilities (and in particular of nuclear power plants) against external impacts in the Federal Republic of Germany 110 (1988) 201

Liemersdorf, H., see Sütterlin 110 (1988) 165

Malvankar, S.V., see Bhanti 110 (1988) 117
Mandl, R.M. and K. Umminger, Water inventory determination using differential pressure cells 110 (1988) 55
Manno, V.P., see Seong 110 (1988) 33
Mewes, D., see Spatz 110 (1988) 413
Morioka, T., see Takizawa 110 (1988) 385
Mueller, G.E., Transient analytical temperature distributions in cylindrical packed beds volumetrically heated by radiogenic decay energy 110 (1988) 329

Oka, Y., see Kataoka 110 (1988) 99

Packer, D., see Husseiny 110 (1988) 277
Peppler, W. and H. Will, Analysis of hexcan failures occuring during the simulation of severe accidents in liquid metal cooled reactors 110 (1988) 73
Pradlwarter, H.J., see Schuëller 110 (1988) 221

Raghunath, B., see Bhanti 110 (1988) 117
Ravindra, M.K., Gross design and construction errors, and seismic risk studies 110 (1988) 255
Reichart, G., How to reduce design and construction errors 110 (1988) 251
Riera, J.D., see Bruna 110 (1988) 153
Rodriguez, R.J., see Husseiny 110 (1988) 277

Sabri, Z.A., see Husseiny 110 (1988) 277
Saijou, N., see Takizawa 110 (1988) 385
Schmidt, T., see Brückner-Foit 110 (1988) 361
Schuëller, G.I., N.C. Hampl and H.J. Pradlwarter, Fragility of primary piping of nuclear power plants 110 (1988) 221
Schulz, H., Comments on the probability of leakage in piping systems as used in PRAs 110 (1988) 229
Seong, P.H., V.P. Manno and M.W. Golay, Application of a power plant simplification methodology: The example of the condensate feedwater system 110 (1988) 33
Shang, H., see Hsieh 110 (1988) 17
Snoek, C.W. and L.K.H. Leung, A model for predicting diabatic pressure drops in multi-element fuel channels 110 (1988) 299
Somasundaram, S., see Bhanti 110 (1988) 117
Soundalgekar, V.M., H.S. Takhar and N.V. Vighnesam, Combined free and forced convection flow past a semi-infinite vertical plate with variable surface temperature 110 (1988) 95
Spatz, R. and D. Mewes, The influence of walls and upper tie plate slots on the fooding mechanism in fuel elements with and without heat transfer between steam and water 110 (1988) 413
Steiner, H. and M. Balourdet, The steel behaviour in CABRI experiments. Interpretation of post test examinations 110 (1988) 361
Sütterlin, L. and H. Liemersdorf, Probabilistic risk assessment of nuclear power plants for seismic events in the Federal Republic of Germany 110 (1988) 165
Sütterlin, L., see Liemersdorf 110 (1988) 201

Takhar, H.S., see Soundalgekar 110 (1988) 95
Takizawa, Y., T. Hayashi, N. Saijou and T. Morioka, A post trip operational guidance system for BWR plants 110 (1988) 385
Theodoropoulos, J., see Brückner-Foit 110 (1988) 361
Thomas, R.M., Methods for calculating the conduction-controlled rewetting of a cladded rod 110 (1988) 1

Umminger, K., see Mandl 110 (1988) 55

Vighnesam, N.V., see Soundalgekar 110 (1988) 95

Wang, C.H. and V.K. Dhir, An experimental investigation of multidimensional quenching of a simulated core debris bed 110 (1988) 61
Wang, Z., see Di Marzo 110 (1988) 107
Will, H., see Peppler 110 (1988) 73

Xu, Z., On the stochastic modeling of earthquake strong ground motion 110 (1988) 171

Yen, B.C., Flood hazards for nuclear power plants 110 (1988) 213

Nuclear Engineering and Design 110 (1989) 425–426
North-Holland, Amsterdam



# SUBJECT INDEX TO VOLUME 110

**Thermal and fluid design**

Methods for calculating the conduction-controlled rewetting of a cladded rod, R.M. Thomas 110 (1988) 1
Solution of boundary value heat conduction problems with variable convective coefficients by a boundary condition dissection method, C.K. Hsieh and M. Shang 110 (1988) 17
Application of a power plant simplification methodology: The example of the condensate feedwater system, P.H. Seong, V.P. Manno and M.W. Golay 110 (1988) 33
Reactor safety implications of pump run-up EBR-II, W.K. Lehto, E.M. Dean and R.M. Freyer 110 (1988) 47
Water inventory determination using differential pressure cells, R.M. Mandl and K. Umminger 110 (1988) 55
An experimental investigation of multidimentional quenching of a simulated core debris bed, C.H. Wang and V.K. Dhir 110 (1988) 61
Analysis of hexcan failures occurring during the simulation of severe accidents in Liquid Metal cooled Reactors, W. Peppler and H. Will 110 (1988) 73
Combined free and forced convection flow past a semi-infinite vertical plate with variable surface temperature, V.M. Soundalgekar, H.S. Takhar and N.V. Vighnesam 110 (1988) 95
Neutronic and thermodynamic feasibility of $^{233}UF_6$–$^{237}NpF_6$ fueled transmutation reactor, K. Kataoka, K. Furuta, Y. Oka and S. Kondo 110 (1988) 99
The phenomenology of a small break LOCA in a complex thermal hydrualic loop, M. di Marzo, K.K. Almenas, Y.Y Hsu and Z. Wang 110 (1988) 107
A model for predicting diabatic pressure drops in multi-element fuel channels, C.W. Snoek and L.K.H. Leung 110 (1988) 299
Long-term recovery of pressurized water reactors following a large break loss-of-coolant accident 110 (1988) 313
Transient analytical temperature distributions in cylindrical packed beds volumetrically heated by radiogenic decay energy, G.E. Mueller 110 (1988) 329
Measurements of neutron induced nucleation, A.A. Kendoush 110 (1988) 349
The influence of walls and upper tie plate slots on the fooding mechanism in fuel elements with and without heat transfer between steam and water, R. Spatz and D. Mewes 110 (1988) 413

**Engineering mechanics**

Nonlinear seismic analysis of a graphite reactor core, W.L. LaFramboise and T.P. Desmond 110 (1988) 131
Towards the simultaneous generation of the three components of the seismic acceleration on rock surface, R. Bruna and J.D. Riera 110 (1988) 153
On the stochastic modeling of earthquake strong ground motion, Xu Z. 110 (1988) 171
A comparison of the PRAISE code and the PARIS code for the evaluation of the failure probability of crack-containing components, A. Brückner-Foit, Th. Schmidt and J. Theodoropoulos 110 (1988) 395

**Materials properties**

The steel behaviour in CABRI experiments: Interpretation of post test examinations, H. Steiner and M. Balourdet 110 (1988) 361

**Mechanical design, component fabrication and testing**

Fragility of primary piping of nuclear power plants, G.I. Schuëller, N.C. Hanpl and H.J. Pradlwarter 110 (1988) 221
Comments on the probability of leakage in piping systems as used in PRAs, H. Schultz 110 (1988) 229
Seismic design concept using methods of probabilistic structural mechanics, P. Jehlicka 110 (1988) 247

How to reduce design and construction errors, G. Reichart 110 (1988) 251
Gross design and construction errors, and seismic risk studies, M.K. Ravindra 110 (1988) 255
A comparison of the PRAISE code and the PARIS code for the evaluation of the failure probability of crack-containing components, A. Brückner-Foit, Th. Schmidt and J. Theodoropoulos 110 (1988) 395

**Reactor concepts and systems analysis**

Probabilistic risk assessment of nuclear power plants for seismic events in the Federal Republic of Germany, L. Sütterlin and H. Liemersdorf 110 (1988) 165
Reduction of the dynamic response by aircraft crash on building structures, N.J. Krutzik 110 (1988) 191
Concepts and design criteria for the protection of nuclear facilities (and in particular of nuclear power plants) against external impacts in the Federal Republic of Germany, H. Liemersdorf and L. Sütterlin 110 (1988) 201
Probability analysis of an airdraft crash to a nuclear power plant, T. Kobayashi 110 (1988) 207
Flood hazards for nuclear power plants, Yen, B.C. 110 (1988) 213
Fragility of primary piping of nuclear power plants, G.I. Schuëller, N.C. Hampl and H.J. Pradlwarter 110 (1988) 221
Comments on the probability of leakage in piping systems as used in PRAs, H. Schultz 110 (1988) 229
A post trip operational guidance system for BWR plants, Y. Takizawa, T. Hayashi, N. Saijou and T. Morioka 110 (1988) 385

**Layout of nuclear facilities**

Preventive fire protection in nuclear power plants, K. Kordina and R. Dobbernack 110 (1988) 185

**Structural design, construction methods and testing**

Assessment of the response of reinforced concrete structural members to aircraft crash impact loading, K. Brandes 110 (1988) 177
Seismic design concept using methods of probabilistic structural mechanics, P. Jehlicka 110 (1988) 247

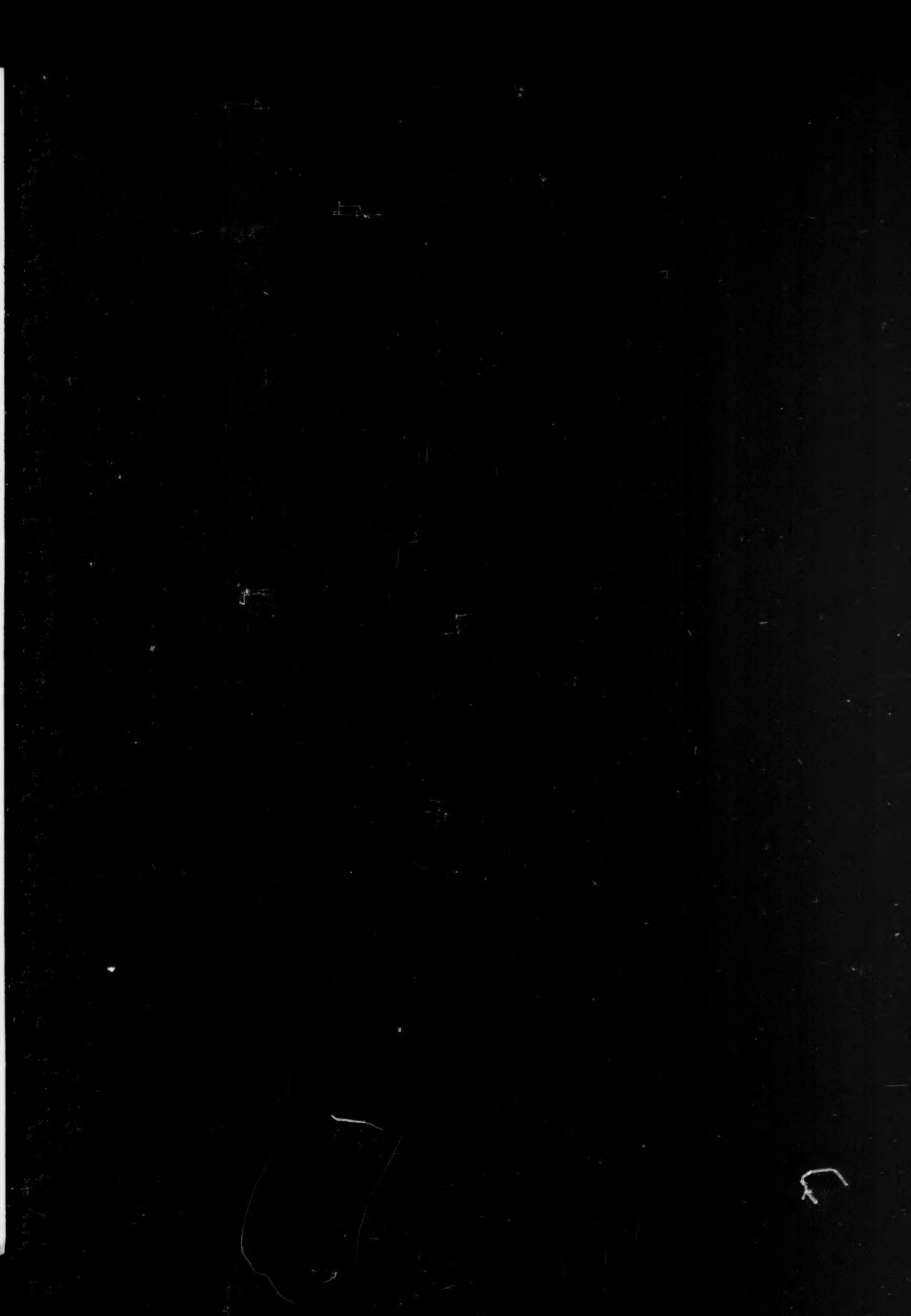
How to reduce design and construction errors, G. Reichart 110 (1988) 251
Gross design and construction errors, and seismic risk studies, M.K. Ravindra 110 (1988) 255

**System control and controls**

System ergonomics as an approach to improve human reliability, H. Bubb 110 (1988) 233
Seismic-load-induced human errors and countermeasures using computer graphics in plant-operator communication, F. Hara 110 (1988) 265

**Core physics and fuel design**

The steel behaviour in CABRI experiments: Interpretation of post test examinations, H. Steiner and M. Balourdet 110 (1988) 361

**Nuclear safety**

Reactor safety implications of pump run-up tests in EBR-II, W.K. Lehto, E.M. Dean and R.M. Freyer 110 (1988) 47
Analysis of hexcan failures occurring during the simulation of severe accidents in Liquid Metal cooled Reactors, W. Peppler and H. Will 110 (1988) 73
The phenomenology of a small break LOCA in a complex thermal hydraulic loop, M. de Marzi, K.K. Almenas, Y.Y. Hsu and Z. Wang 110 (1988) 107
Aerosol release factor for Pu as a consequence of ion exchange resin fire in the process cell of a fuel reprocessing plant, D.P. Bhanti, S.V. Malvankar, P. Kotrappa, S. Somasundaram, B. Raghunath and A.M. Curtay 110 (1988) 117
Operating procedure automation to enhance safety of nuclear power plants, A.A. Husseiny, Z.A. Sabri, D. Packer, J.W. Holmes, S.K. Adams and R.J. Rodriguez 110 (1988) 277
Long-term recovery of pressurized water reactors following a large break loss-of-coolant accident, C.D. Fletcher and R.A. Callow 110 (1988) 313
Transient analytical temperature distributions incylindrical packed beds volumetrically heated by radiogenic decay energy, G.E. Mueller 110 (1988) 329